

sCO₂ Cycle Modeling at ANL: Performance, Optimization, Control, and Dynamics

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Motivation and Approach

- sCO₂ Cycle modeling at Argonne started in 2001
 - NERI + Ph.D. Dissertation -> Gen IV -> AFCI -> ARC -> SMR -> ART
- Needed answers to fundamental questions
 - What is the cycle efficiency?
 - Is cycle better than steam?
 - What it takes to achieve the performance?
 - Can you control the cycle close to critical point?
 - What is transient response of the cycle? For nuclear power plants, need to calculate operational transients and postulated accidents for safety evaluation
- Solution: create a first-principal but realistic code
 - Ability to modify, improve, and extend the code as knowledge on the cycle grew
 - Be able to couple to Argonne reactor analysis codes
 - Sufficient detail to serve as “simulation experiments”

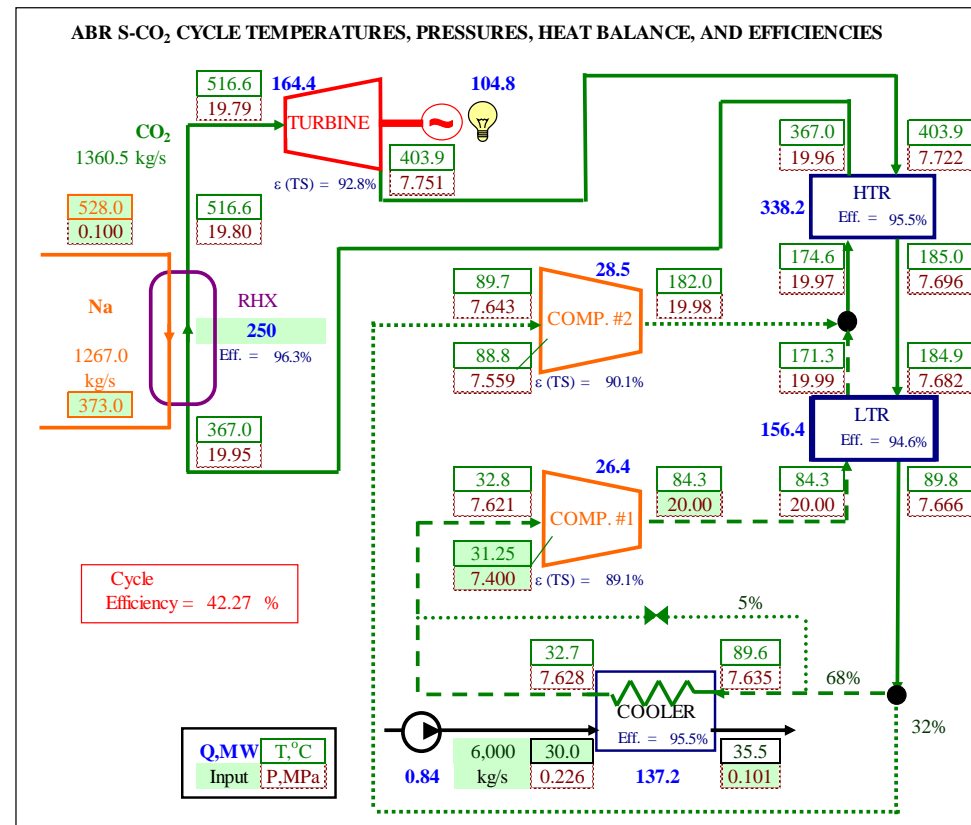
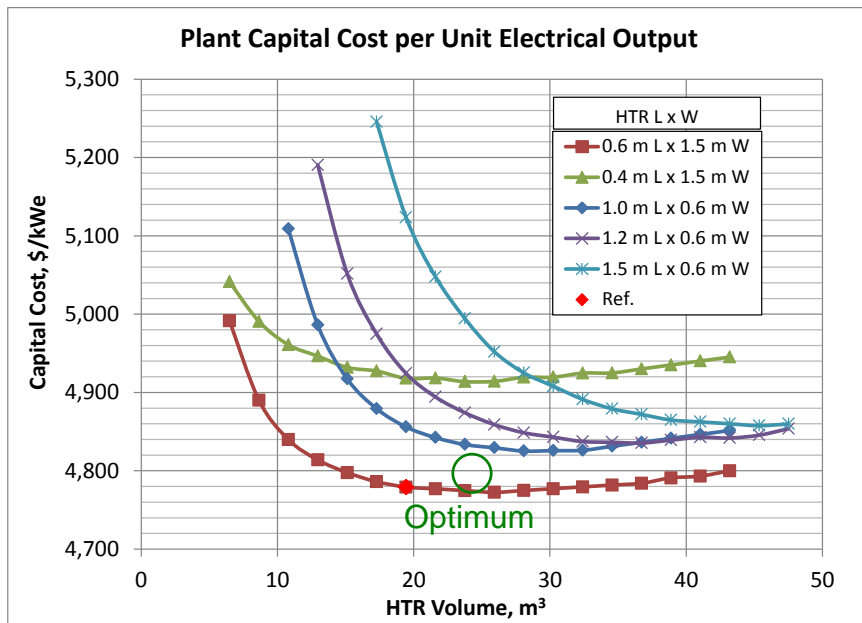
ANL Plant Dynamics Code (PDC)

- Specifically developed for analysis of S-CO₂ cycle
 - One-dimensional system level transient analysis code
 - Targets the specific features of the cycle
 - Operation close to the critical point
 - Recompression cycle (if needed)
 - Real CO₂ properties
 - Property variations in HX's and turbomachinery
 - No simplifying ideal gas assumptions
 - Compressibility effects
- Incorporates S-CO₂ cycle control mechanisms and logic
- Incorporates steady-state design code to determine cycle initial conditions
- Design and performance subroutines for both turbine and compressor
- Coupled to SAS4A/SASSYS-1 that performs reactor dynamic analysis
- Validation of the PDC against the SNL RCBC and BMPC IST data is ongoing



Steady-State Performance & Optimization

- Calculate design performance for the cycle (efficiency) and each component (effectiveness, pressure drop)
- Trade-off and optimization
 - Operating conditions (e.g., pressures)
 - HX size and performance vs cost



Transient Analysis

- For **nuclear applications**, transient performance of the cycle is also important
 - Cycle control and load following
 - E.g., integration with autonomous reactor control
 - Accident conditions – how they affect **reactor safety**
 - E.g., Postulated CO₂ pipe break = A nuclear power plant design basis accident
- Transient part of the PDC also developed to account for specifics of operating near the critical point
 - Properties variation in HXs and turbomachinery
 - No ideal gas assumptions
 - Compressibility effects
 - Equations are written to account for properties**

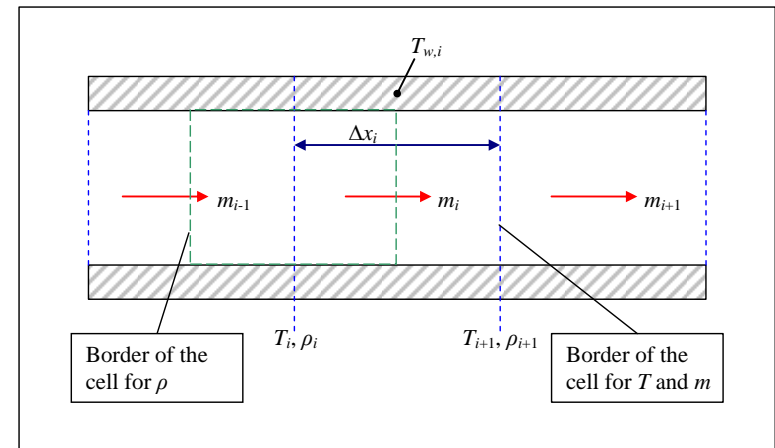
$$\frac{\partial h_{i+1}}{\partial t} = \frac{1}{M_i} \frac{\dot{m}_i + \dot{m}_{i-1}}{2} (h_i - h_{i+1}) + \frac{\Delta x_i N_t}{M_i \text{res}_{w,CO_2}} \left(T_{w,i} - \frac{T_i + T_{i+1}}{2} \right)$$

$$\frac{\partial \rho_i}{\partial t} = \frac{1}{A \left(\frac{\Delta x_i}{2} + \frac{\Delta x_{i-1}}{2} \right)} (\dot{m}_i - \dot{m}_{i-1})$$

$$\frac{\partial \dot{m}_i}{\partial t} = \frac{A}{\Delta x_i} (p_i - p_{i+1}) - \frac{2 f_i \Delta x_i}{M_i D_h} \dot{m}_i^2$$

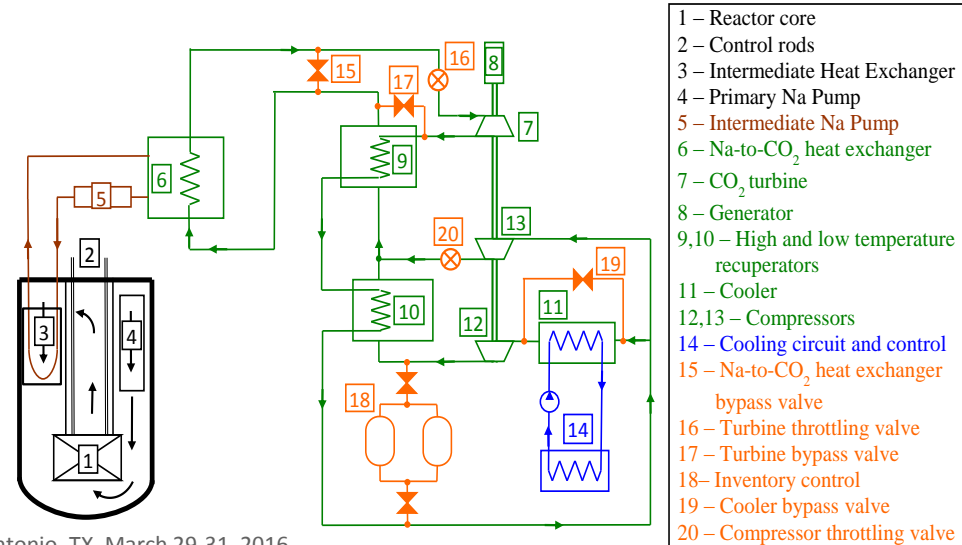
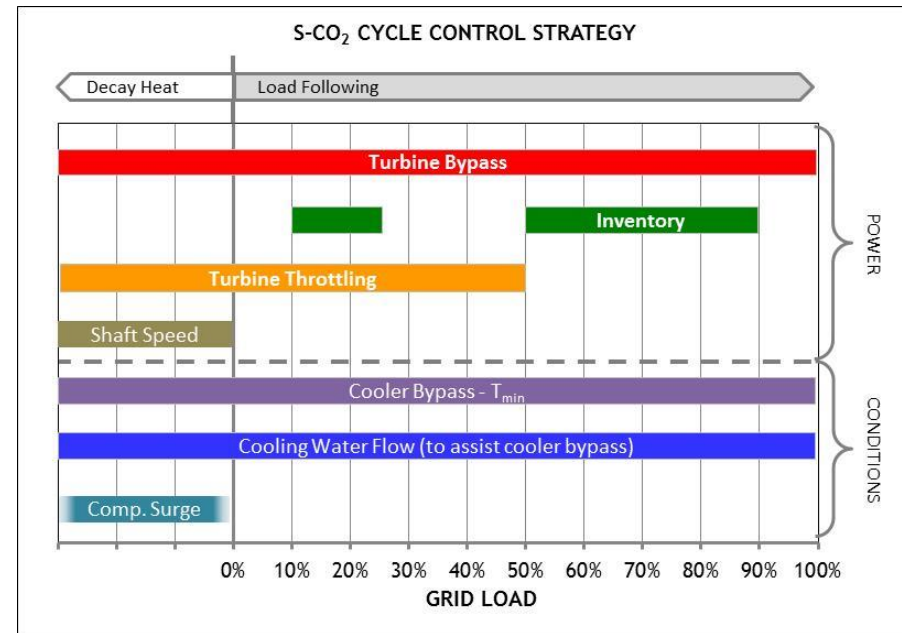
$$\frac{\partial h}{\partial t} = \left(\frac{\partial h}{\partial T} \right)_\rho \frac{\partial T}{\partial t} + \left(\frac{\partial h}{\partial \rho} \right)_T \frac{\partial \rho}{\partial t}$$

$$\frac{\partial p}{\partial t} = \left(\frac{\partial p}{\partial T} \right)_\rho \frac{\partial T}{\partial t} + \left(\frac{\partial p}{\partial \rho} \right)_T \frac{\partial \rho}{\partial t}$$



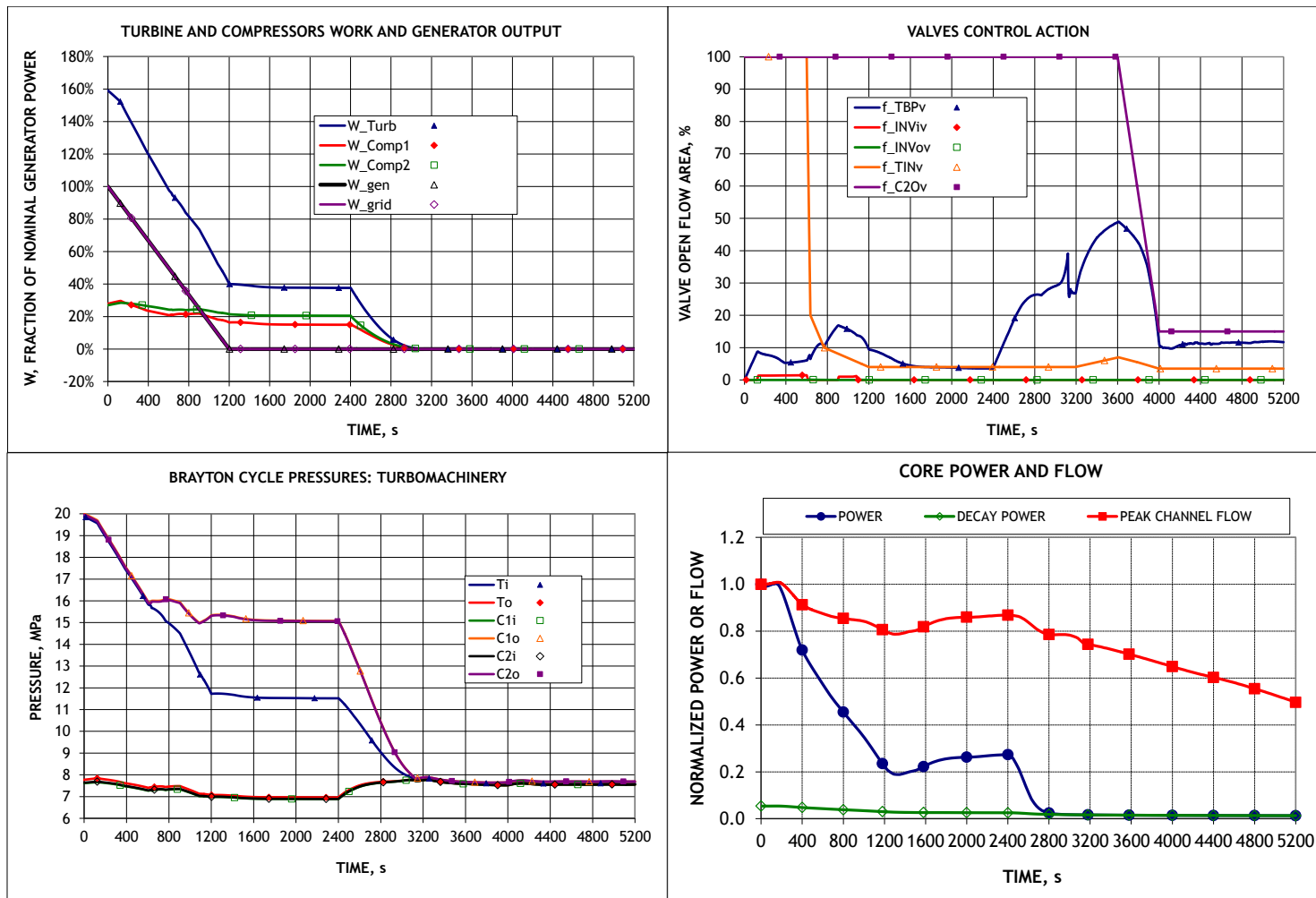
Control System Development

- Significant effort was devoted to developing cycle control approach
 - Grid load following
 - Decay heat removal mode
 - Controllability near the critical point
 - Interaction with reactor control
- Combination of various control mechanisms is required



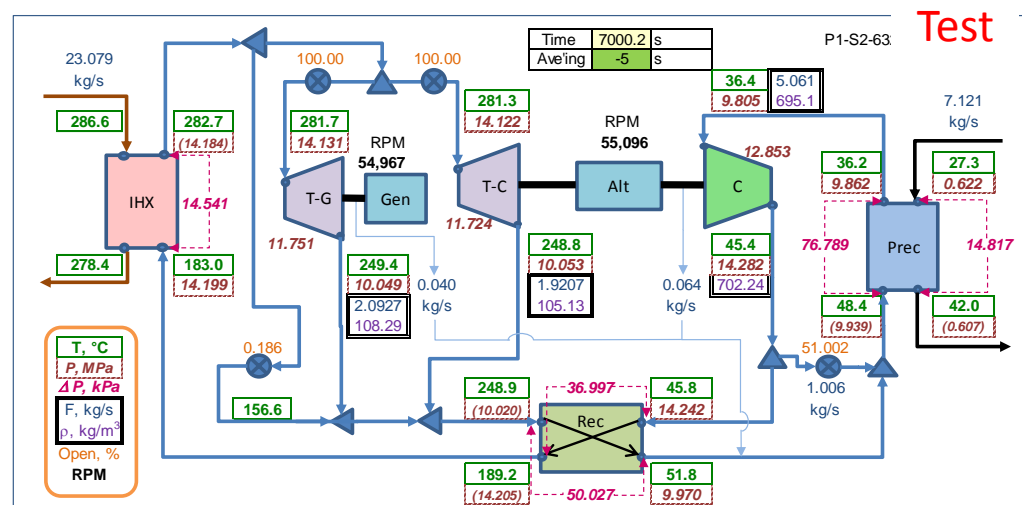
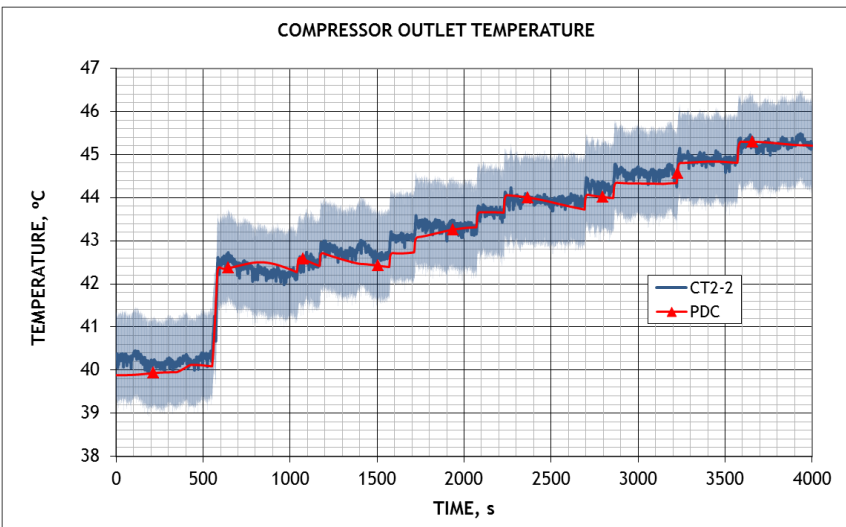
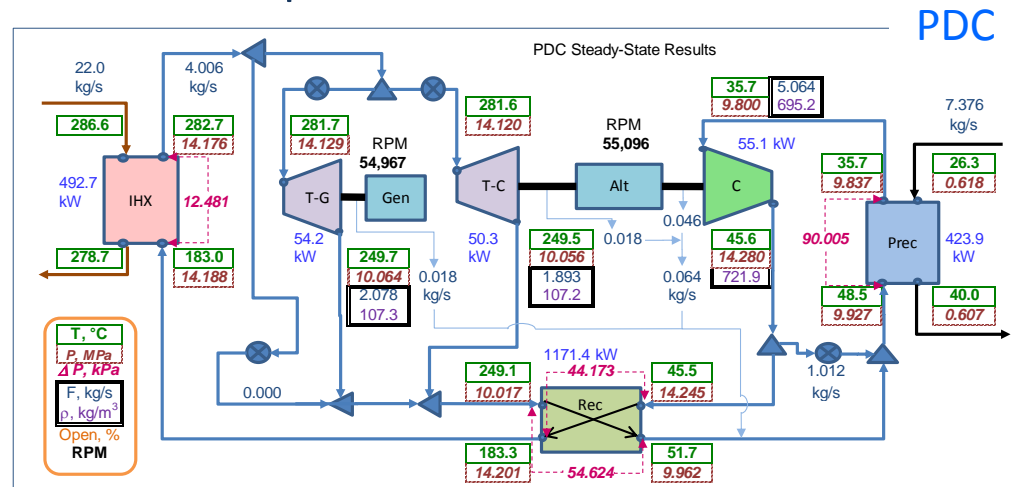
Example of Transient Calculations

- Load following from 100% to 0% followed by disconnection from the grid and transition to decay heat removal mode



PDC Validation

- Validation is an essential part of code development
 - PCHE tests at Argonne
 - SNL Recompression Closed Brayton Cycle facility
 - BPMC Integrated System Test
- Both steady-state and transient data



Future Plans

- Continue PDC validation moving on to data from larger-scale facilities (STEP, etc.)
- Application of the PDC to dry air-cooled sCO₂ cycles
- Investigation of potential benefits of Model-Based Predictive Control and other advanced control methodologies
- Applications to other heat sources than nuclear (fossil, solar)

