

sCO₂ Cycle Modeling at ANL: Performance, Optimization, Control, and Dynamics

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Motivation and Approach

- sCO₂ Cycle modeling at Argonne started in 2001
 - NERI + Ph.D. Dissertation -> Gen IV -> AFCI -> ARC -> SMR -> ART
- Needed answers to fundamental questions
 - What is the cycle efficiency?
 - Is cycle better than steam?
 - What it takes to achieve the performance?
 - Can you control the cycle close to critical point?
 - What is transient response of the cycle? For nuclear power plants, need to calculate operational transients and postulated accidents for safety evaluation
- Solution: create a first-principal but realistic code
 - Ability to modify, improve, and extend the code as knowledge on the cycle grew
 - Be able to couple to Argonne reactor analysis codes
 - Sufficient detail to serve as "simulation experiments"

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ANL Plant Dynamics Code (PDC)

- Specifically developed for analysis of S-CO₂ cycle
 - One-dimensional system level transient analysis code
 - Targets the specific features of the cycle
 - Operation close to the critical point
 - Recompression cycle (if needed)
 - Real CO₂ properties
 - Property variations in HX's and turbomachinery
 - No simplifying ideal gas assumptions
 - Compressibility effects
- Incorporates S-CO₂ cycle control mechanisms and logic
- Incorporates steady-state design code to determine cycle initial conditions
- Design and performance subroutines for both turbine and compressor
- Coupled to SAS4A/SASSYS-1 that performs reactor dynamic analysis
- Validation of the PDC against the SNL RCBC and BMPC IST data is ongoing

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Steady-State Performance & Optimization

- Calculate design performance for the cycle (efficiency) and each component (effectiveness, pressure drop)
- ABR S-CO2 CYCLE TEMPERATURES, PRESSURES, HEAT BALANCE, AND EFFICIENCIES Trade-off and optimization 516.6 104.8 164.4 19.79 **Operating conditions FURBINE CO**₂ 367.0 1360.5 kg/s 403.9 19.96 (e.g., pressures) 7.751 ε (TS) = 92.8% 338.2 528.0 516.6 0.100 19.80 HX size and performance vs cost 174.6 28.5 89.7 182.0 19.97 RHX 19.98 Na 7.643 250 **Plant Capital Cost per Unit Electrical Output** Eff. = 96.3% 1267.0 88.8 171.3 19.99 $7.559 \epsilon (TS) = 90.1\%$ kg/s 5,300 HTR L x W 367.0 156.4 19.95 5,200 26.4 32.8 84.3 84.3 🛨 0.4 m L x 1.5 m W 20.00 20.00 7.621 Capital Cost, \$/kWe -1.0 m L x 0.6 m W 5,100 31.25 Cycle 7.400 ϵ (TS) = 89.1% Efficiency = 42.27 % 5% Ref. 5,000 32.7 89.6 7.628 7.635 68% 4.900 COOLER Eff. = 95.5% **Q,MW** T,^oC 6.000 30.0 Input P,MPa 4,800 0.84 kg/s 0.226 137.2 0.101Optimum

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40

30

4,700

0

10

20

HTR Volume, m³

403.9 7.722

185.0

7.696

184.9

7.682

89.8

7.666

32%

HTR

LTR

Eff. = 94.6%

Eff. = 95.5%

Transient Analysis

- For nuclear applications, transient performance of the cycle is also important
 - Cycle control and load following
 - E.g., integration with autonomous reactor control
 - Accident conditions how they affect reactor safety
 - E.g., Postulated CO₂ pipe break = A nuclear power plant design basis accident
- Transient part of the PDC also developed to account for specifics of operating near the critical point
 - Properties variation in HXs and turbomachinery
 - No ideal gas assumptions
 - Compressibility effects
 - Equations are written to account for properties

$$\frac{\partial h_{i+1}}{\partial t} = \frac{1}{M_i} \frac{\dot{m}_i + \dot{m}_{i-1}}{2} (h_i - h_{i+1}) + \frac{\Delta x_i N_t}{M_i res_{w, CO_{2i}}} \left(T_{w,i} - \frac{T_i + T_{i+1}}{2} \right)$$

$$\frac{\partial \rho_i}{\partial t} = \frac{1}{A\left(\frac{\Delta x_i}{2} + \frac{\Delta x_{i-1}}{2}\right)} \left(\dot{m}_i - \dot{m}_{i-1}\right)$$

$$\frac{\partial \dot{m}_i}{\partial t} = \frac{A}{\Delta x_i} \left(p_i - p_{i+1} \right) - \frac{2 f_i \Delta x_i}{M_i D_h} \dot{m}_i^2$$

$$\begin{cases} \frac{\partial h}{\partial t} = \left(\frac{\partial h}{\partial T}\right)_{\rho} \frac{\partial T}{\partial t} + \left(\frac{\partial h}{\partial \rho}\right)_{T} \frac{\partial \rho}{\partial t} \\ \frac{\partial p}{\partial t} = \left(\frac{\partial p}{\partial T}\right)_{\rho} \frac{\partial T}{\partial t} + \left(\frac{\partial p}{\partial \rho}\right)_{T} \frac{\partial \rho}{\partial t} \end{cases}$$



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Control System Development

- Significant effort was devoted to developing cycle control approach
 - Grid load following
 - Decay heat removal mode
 - Controllability near the critical point
 - Interaction with reactor control
- Combination of various control mechanisms is <u>required</u>



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Example of Transient Calculations

 Load following from 100% to 0% followed by disconnection from the grid and transition to decay heat removal mode



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PDC Validation

- Validation is an essential part of code development
 - PCHE tests at Argonne

COMPRESSOR OUTLET TEMPERATURE

- SNL Recompression Closed
 Brayton Cycle facility
- BPMC Integrated System Test
- Both steady-state and transient data

47

46

45

44

43

42

41

40

39

38

0

500

1000

1500

2000

TIME, s

2500

ů

TEMPERATURE,



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3000

-CT2-2

3500

PDC

Future Plans

- Continue PDC validation moving on to data from larger-scale facilities (STEP, etc.)
- Application of the PDC to dry air-cooled sCO₂ cycles
- Investigation of potential benefits of Model-Based Predictive Control and other advanced control methodologies
- Applications to other heat sources than nuclear (fossil, solar)



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